Graphite Research

THE QUEEN'S ANNIVERSARY PRIZES FOR HIGHER AND FURTHER EDUCATION 2011 & 2013

• Why?

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- Core of AGRs; irradiation damage during operation leads to weight loss, change in dimensions & properties, stress and cracking
- Life-limiting component of reactors
 - See recent (BBC, Times) and upcoming news items
- Current research activity
 - Funded by industry (EDF), regulator (ONR), EPSRC, TSB, EU
 - Plant life extension (PLEX) and future systems
 - Fundamentals of irradiation damage
 - Irradiation creep (mitigating stress development)
 - Strength and fracture
 - Waste treatment

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Irradiation damage

- FunGraph (EPSRC)
 - Fundamentals of Current and Future Uses of Nuclear Graphite
 - Consortium of Huddersfield, Leeds, Manchester, Nottingham, Salford, Surrey universities
- Mechanistic understanding of irradiation effects
 - Ab initio; kinetic Monte Carlo; dislocation models of defects
 - Electron microscopy
 - Model (thick graphene) and irradiated material
 - Evolution of damage in situ
 - Neutron and synchrotron x-ray studies
 - Structural changes; crack and void closure; effects of loading Range of model and irradiated materials





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Neutron irradiated samples

- Manchester's samples
 - Oldbury (PGA 5dpa)
 - BEPO channel 16 (PGA)
- 80 kV





FunGraph meeting 29th – 31st October 2012 Huddersfield

Microstructure: Irradiated graphite

Irradiated AGR Near surface 120E20 EDND Irradiated AGR Near surface 120E20 EDND



Imaged at Central Labs, NNL

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Irradiation Creep

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- TSB-funded, EDF-lead
 - Influence of Irradiation Creep on Plant Life Optimisation
 - Manchester and Surrey universities
 - NRG, Fraser-Nash, AMEC
- Mechanistic understanding of irradiation creep
 - Defect structures and mobility under load
 - Configuration; energies of formation
 - Microstructural changes
 - Role of interfaces/boundaries
 - Stress relaxation

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Unbound and bound configurations of the $V^{2,2}_{bb}$ interlayer di-vacancy.



Microstructure-based FE models for irradiation creep







- QUBE (EPSRC)
 - Oxford, Bristol and Manchester universities



Microstructure-Sensitive Component-Scale Fracture Modelling



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ANNIVERSARY PRIZES Cellular FE model: 2011 St 2013 2011 & 2013 Meshfree linking of microstructure-dependent heterogeneous model and coarse FE component model

The CA model may be used as microstructure dependent input, providing properties of coarser "cells" in the Cellular FE model

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- HSE(NII) established the AGR Brick Cracking Network (AGR BCN) in 2008 to:
 - develop an improved understanding of cracking observed in AGR graphite bricks and develop a predictive capability
 - secure, maintain and develop sources of independent advice
 - ensure NII is well informed on key technical issues and therefore able to make sound regulatory decisions
- AGR BCN is a tripartite programme
 - The University of Birmingham, The University of Manchester, Health & Safety Laboratory
- Two aspects under investigation
 - crack driving force parameters (HSL & UoM)
 - materials resistance parameters (UoB & HSL)

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Introduction



- Emulator approach
 - applied successfully for dimensional change behaviour
 - inspection data
 - constitutive equations
 - calibrated dimensional changes
 - inert similar to baseline
 - reduced effect of oxidation







Modelling approach



- Two TSB projects; both EDF-lead
 - Fracture of Graphite Fuel Bricks
 - University of Manchester, EDF NG, EDF R&D
- Modelling crack propagation in fuel bricks
 - XFEM, energy release rates
 - Comparison with experiment
- Statistical models of core normal and seismic behaviour
- Mimic irradiation damage by bromination
 - No external driving forces for crack growth (cf in service)
 - Criteria for crack initiation as function of local microstructure



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Tomographic image of brominated grapite



Plate 5 Prismatic torsion specimen after failure (After Brokenshire 1995)



Comparison of crack paths with prediction

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- Influence of Creep and Geometry on Strength of Irradiated Graphite Components
 - University of Manchester, EDF NG
- Valid fracture data on irradiated material
 - Large trepanned specimens at reactor end-of-life
 - Ex-Oldbury
 - effects of notch geometries on properties
- Crack initiation and propagation
- Comparison with AGR installed sets
- Thermal creep as basis for irradiation creep
- Effects of creep on mechanical/fracture behaviour

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